

DTT Industry Day



# **DTT project overview**

#### Villa Mondragone, Monte Porzio Catone (Rome), Italy – 14/12/2018

Aldo Pizzuto - ENEA - Director fo Department for Fusion and Technology for Nuclear Safety and Security

# Why DTT?

JET (in operation since 1983): designed to study plasma behaviour in conditions and dimensions approaching those required in a fusion reactor (including D-T operation)

ITER (under construction): reactors-scale international experiment designed to deliver ten times more power than it consumes (burning plasma with  $Q \sim 10$ )

# DTT: a facility conceived to develop and test *integrated,* controllable power *exhaust solutions* for DEMO

DEMO (predesign activities): expected to be the first fusion plant to provide electricity to the grid





# The DTT added value

DTT is a new device, where the modern technologies can be adopted and further developed; the presently operating Tokamaks were designed about 40 years ago

It is reasonable affirm that by 2025 most of the plasma experiments built in the '80 are shutdown and the experimental plasma physics activities are carried out on a few (modern) machines

DTT construction will keep industry linked to fusion field

DTT would be the ideal training device to grow the new generations needed for feeding ITER and, subsequently DEMO, with skilled people





### Titolo della slide



# The Divertor Tokamak test facility

#### DTT has most of the features of a next generation tokamaks



Fusion Engineering and Design, Special Issue: DTT

https://www.sciencedirect.com/science/journal/09203796/122

http://www.dtt-project.enea.it

Soon green book

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# **DTT main parameters**





#### **Main Parameters**

R (m)	2.10
a (m)	0.65
lp (MA)	5.5
BT (T)	6.0
Vpl (m <sup>3</sup> )	33.0
Padd (MW)	45.0



#### **Procurement and services**

#### **DTT main procurements:**

#### 1.Superconducting Magnets:

Strands: Ni3Sn and NiTi

Cables

Magnets (coils+casings) External structure

#### 2.Vessel/In-Vessel

Vacuum Chamber First Wall Divertor

### 3. Power Supplies:

CS, PF, TF & protection systems Additional heating Auxiliaries Distribution systems

#### 4. Radiofrequency:

Ion Cyclotron: 4 MW at 90 MHz Electron Cyclotron: 14 MW at 170 GHz Neutral Beam Injector 7 MW

5. Cryocooler: 10 kW at 4.6 K

### 6. Control

- 7. Remote maintenance In vessel ex vessel
- 8. Buildings
- 9. Assembly





# **DTT Magnets**

### **DTT – Magnet System**

#### **Main Features**



18 <u>Toroidal Field coils</u>
Nb<sub>3</sub>Sn
6 <u>Central Solenoid module coils</u>
Nb<sub>3</sub>Sn
6 <u>Poloidal Field coils</u>
2 Nb<sub>3</sub>Sn
4 NbTi





# **DTT Magnets**

### DTT – CS solenoid





MF: 4 x 22 = 88 turns

#### LF:

8 x 26 = 208 turns

I\_op: 28.0 kA





## **Precompression rings**

Two couple rings made of glass-epoxy vacuum impregnated







# **DTT Vacuum Vessel**

#### 18 sectors – double wall structure – AISI 316 LN – 5 ports/sector – 2 NBI eq. port inclined 30°







# **DTT Vacuum Vessel**





## **DTT Vacuum Vessel**

# Two modules (three sectors) will be modified to accommodate two co-tangential NNBI.



The beams are injected at 40° relative to the magnetic axis



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# **DTT Divertor & First Wall**



Full scale full W ITER divertor inner vertical target plasma facing units





#### Possible provisional divertor





DTT Industry Day - 14 December 2018 Innovative Nuclear Systems Technology Development. The Italian's Contribution IAEA General Conference (61<sup>st</sup> Session)



# **DTT Divertor & First Wall**



# **First wall**

#### *Two coaxial concepts* under study:

•square cross-section with CuCrZr as structural material circular cross-section without copper

#### In both cases:

IAEA General Conference (61st Session)

#### ≈3 mm Tungsten plasma spray with Functional

Gradient Material interlayer





# **DTT Cryostat**



# **Additional heating: ECRH**

### ECRH: 16 gyrotrons

#### Scheme:

4 Solid State HVPS
8 Gyrotrons (1MW/170GHz/100s) (ITER like)
1 Multi-Beam Quasi Optical TL 90% efficiency

#### Front Steering Antennas:

6 Lines in 1 Equatorial Port2 Lines in 1 Upper Port







# **Additional heating: NNBI**



Beam energy: 350 - 450 keV







# **Additional heating: ICRH**

#### **ICRH: Main features**

60-90 MHz frequency range,3 MW at plasma,4 MW at generators.

#### System architecture

1 HVPS, 2 diacrode-based transmitters, coaxial (9 in.) transmission lines, 2 **antennas**: power density ~2.2 MW/m<sup>2</sup>.







The expected neutron rate in DTT amounts to1.3×10<sup>17</sup> ns<sup>-1</sup> for the H-mode extreme scenario *making remote handling mandatory* in the long range.

However our assumption is that the remote handling will operate also during the machine assembling phase. This will allow to test and commission the remote handling during a phase when it is not at all necessary.





# Power Supply





## **RH strategy**

E



- 6 lower/upper ports for Divertor
- 2 equatorial for FW







#### Investments

The total investment is around 450 million euros + contingency The total duration of the commitment is 6,5 years + 6 months for commissioning

The volume of the tender per year is the following:



Commitments and Payments Distribution





Items	Costs (M€)
Components and Parts (Magnets ~60%; Vessel-InVessel+Cryostat~40%)	210
Magnet power supplies (w/o substation)	60
Additional heating (25 MW)	102
Remote Handling and cooling system	37
Diagnostics & control	11
Infrastructures and Assembly	25
Contingency	25
Convises	20
	30
TOTAL	500





### Time schedule of the main components



Divertor Tokamak Te facility Mon 01/01/24 The vast majority of the procurement will be oriented to the SME's

In a few cases the value of the contract is quite high so the bidders must have sufficient financial capacity

Services are expected to be required during the assembly and operation phase.





For a timely realisation the call must be based on suitable specs based on well assessed design and proven technology

Framework contracts and open procurements are deemed too risky for a timely construction within budget

The call will be typically based on negotiated procedures based on a pre-qualifying Call for interest.





### aldo.pizzuto@enea.it



